

McGuire 2

4Q/2010 Plant Inspection Findings

Initiating Events

Significance: G Sep 30, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to determine the cause and take corrective action to preclude repetition for control room area chilled water system

An NRC-identified Green NCV of 10 CFR 50, Appendix B, Criterion XVI, Corrective Action, was identified for the licensee's failure to determine the cause of a significant condition adverse to quality involving both trains of Control Room Area Chilled Water System (CRACWS) being out of service at the same time. This resulted in insufficient corrective action to preclude repetition. The licensee reopened the root cause investigation to determine the cause and was resolving the high cycle fatigue issue on the hot gas bypass line.

The performance deficiency was more than minor because, if left uncorrected, it had the potential to lead to a more significant safety concern in that failing to identify corrective actions to preclude repetition could result in the loss of safety function of more risk-significant equipment such as emergency diesel generators. This finding was determined to be of very low safety significance (Green) because it did not contribute to both the likelihood of a reactor trip and the likelihood that mitigation equipment or functions would not be available. This finding was associated with the cross-cutting aspect of supervisory and management oversight in the Work Practices component of the Human Performance area because management's establishment of the scope and reviews of the completed root cause evaluation failed to provide adequate oversight to ensure the cause of a significant condition adverse to quality was determined and corrective actions were taken to preclude repetition. [H.4(c)] (Section 40A3)

Inspection Report# : [2010004](#) (*pdf*)

Mitigating Systems

Significance: G Oct 22, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Correct a Condition Adverse to Quality Associated with Emergency Diesel Generators Fuel Transfer System Niagara Flow Meters

The NRC identified a Non-cited Violation (NCV) of 10 CFR 50, Appendix B, Criterion XVI, Corrective Action, for the licensee's failure to correct a condition adverse to quality in that a single vulnerability failure of the fuel transfer (FD) system Niagara flow meters identified in 2003 could potentially restrict fuel flow to the EDGs which would impact their safety function. In addition, these flow meters were identified as a Category A risk component which required preventative maintenance (PM) strategy and no PM or inspection for these flow meters was ever performed. This issue was documented in the corrective action program as PIP M-10-6442 and the license intends to replace the flow meters for 1A EDG and 2A EDG in 2011.

The inspectors concluded that the failure to correct a condition adverse to quality for the FD system flow meters identified in 2003 was a performance deficiency (PD). The PD was more than minor because it was associated with the Equipment Performance attribute of the Mitigating Systems Cornerstone in that it adversely affected the reliability of the EDGs to respond to initiating events to prevent undesirable consequences in that the flow meters could potentially restrict fuel flow to the EDGs which would impact their safety function. The finding was determined to have very low safety significance (Green) because there was no loss of safety function of any EDG train. The inspectors determined that the cross-cutting area of Human Performance, component of Work

Control, and aspect of Work Planning was applicable because the licensee did not incorporate risk insights in their plan work activities to remove this potential single vulnerability failure of Niagara flow meters in a timely manner.
H.3(a) (4OA2)

Inspection Report# : [2010006](#) (pdf)

Significance: SL-IV Sep 30, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to update the UFSAR for a modification to the VG system

An NRC-identified SL-IV NCV was identified when the licensee did not update the Updated Final Safety Analysis Report (UFSAR) for a modification to the emergency diesel generator air start system (VG) on both units. This modification installed cross-connect piping between the two VG receivers on each emergency diesel generator to allow maintaining receiver pressure when an air compressor was out of service. Licensee corrective actions include updating the UFSAR and Design Basis Documents and processing a Technical Specification (TS) change to make the TS applicable to the cross-connected configuration. This violation is in the licensee's corrective action program as PIPs M-10-5299 and M-10-5504.

This performance deficiency was considered as traditional enforcement because not having an updated UFSAR hinders the licensee's ability to perform adequate 10 CFR 50.59 evaluations and can impact the NRC's ability to perform its regulatory function such as license amendment reviews and inspections. This violation was determined to be a SL-IV violation using Section 6.1 of the NRC's Enforcement Policy because the inaccurate information was not used to make an unacceptable change to the facility. Cross-cutting aspects are not assigned to traditional enforcement violations. (Section 1R04)

Inspection Report# : [2010004](#) (pdf)

Significance: SL-IV Sep 30, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to update the UFSAR for new EDG tripping functions

A NRC-identified SL-IV NCV of 10 CFR 50.71(e) was identified when the licensee failed to update the UFSAR following a modification that installed new protective functions for the emergency diesel generators (EDGs). This violation is in the licensee's corrective action program as PIP M-10-05718

This performance deficiency was considered as traditional enforcement because not having an updated UFSAR hinders the licensee's ability to perform adequate 10 CFR 50.59 evaluations and can impact the NRC's ability to perform its regulatory function such as license amendment reviews and inspections. This violation was determined to be a SL-IV violation using Section 6.1 of the NRC's Enforcement Policy because the inaccurate information was not used to make an unacceptable change to the facility. Cross-cutting aspects are not assigned to traditional enforcement violations. (Section 1R17)

Inspection Report# : [2010004](#) (pdf)

Significance: SL-IV Mar 31, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to adequately update the UFSAR for FPP documents incorporated by reference

The inspectors identified a non-cited violation (NCV) for the failure to update the Updated Final Safety Analysis Report (UFSAR) as required by 10 CFR 50.71(e) for the Fire Protection Program (FPP) documents that were incorporated by reference. This issue is in the licensee's corrective action program as Problem Investigation Process Report (PIP) M-10-0655. The licensee intends to either provide the required updates to the referenced documents or incorporate the FPP directly into the UFSAR.

The updated information for the UFSAR was important because it identified the elements of the FPP, fire hazards analysis, and safe shutdown analysis that are a portion of the basis for the FPP. This issue was considered as traditional enforcement because it had the potential for impacting the NRC's ability to perform its regulatory function. This issue is not minor because not having an updated portion of the UFSAR hinders the licensee's ability to perform adequate 50.59 evaluations and can impact the NRC's ability to perform adequate regulatory reviews for

license amendments and inspections. Consequently, it can have a material impact on licensed activities. This issue was considered to meet the criteria for a severity level IV violation in Supplement I of the NRC Enforcement Policy because the information was not used to make an unacceptable change to the facility or procedures. This violation was not screened for associated cross-cutting aspects because it dealt with traditional enforcement. (Section 1R05)

Inspection Report# : [2010002](#) (pdf)

Significance:  Mar 31, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to adequately implement the Fire Protection Program (FPP) for the Standby Shutdown System (SSS)

The inspectors identified a Green NCV of the FPP required by 10 CFR 50.48 and License Condition 2.C.4 for failing to take adequate design control measures associated with the addition of the standby shutdown system (SSS) for both Units. Specifically, the licensee failed to include a fire hazards analysis (FHA) in the FPP for the SSS, and failed to enter the SSS into the quality assurance program (QAP). The licensee performed a functionality assessment for the area where the SSS is located. The licensee intends to add the SSS to the FHA and the QAP. In addition, any previous modifications made to the SSS will be reviewed and corrective action taken as appropriate.

The performance deficiency was greater than minor because it affected the Mitigating Systems Cornerstone objective of availability, reliability, and capability of the post-fire safe shutdown (SSD) systems and is associated with the design control and protection against external factors (fire) attributes. Specifically, there was no FHA that demonstrated the availability and capability that at least one SSD train would be free of fire and capable of performing safe shutdown as required by 10 CFR 50.48, (a)(2)(iii). The issue was determined to be of very low safety significance (Green) using IMC 0609, Appendix F, Attachment 1, based on the fact that the categories of Fire Prevention and Administrative Controls, and post-fire SSD, were evaluated as having low degradation. There was no cross-cutting aspect associated with this performance deficiency because it was not representative of current licensee performance. (Section 1R18)

Inspection Report# : [2010002](#) (pdf)

Significance:  Mar 31, 2010

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to flow test nuclear service water "A" train standby nuclear service water pond (SNSWP) supply header at maximum design.

A self-revealing Green NCV of 10 CFR 50, Appendix B, Criterion XI, Test Control, was identified for the licensee's failure to flow test the Nuclear Service Water System (NSWS) "A" Train Standby Nuclear Service Water Pond (SNSWP) unit common supply header at maximum design flow. The licensee entered this issue into their corrective action program as PIP M-09-2216 and has taken corrective actions to increase the minimum required flow velocity, frequency, and duration of the "A" Train SNSWP unit common supply header test procedure.

The finding was more than minor because it affected the cornerstone attributes of "protection against external events" and "equipment performance" and the Mitigating Systems objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Specifically, inadequate flushing of the "A" Train SNSWP unit common supply header led to ineffective flushes and the accumulation of corrosion products which challenged the design function of the NSWS system. This finding was evaluated using IMC 0609, Attachment 4, Phase I - Initial Screening and Characterization of Findings, to determine the safety significance. Since the finding was related to a seismic initiating event, a Phase III was required to be performed by an NRC Senior Risk Analyst. The Phase III analysis calculated the risk increase to be less than 1E-7 for both conditional core damage probability and conditional large early release probability, resulting in a determination of very low risk significance (Green). This performance deficiency was associated with the cross-cutting aspect of complete, accurate and up-to-date design documentation and procedures [H.2(c)] as described in the Resources component of the Human Performance cross-cutting area. (Section 4OA3.1)

Inspection Report# : [2010002](#) (pdf)

Barrier Integrity

Emergency Preparedness

Occupational Radiation Safety

Significance: G Jun 30, 2010

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to barricade, conspicuously post, and adequately control access to a HRA

•Green. A self-revealing NCV of TS 5.7.1 was identified for the failure of the licensee to barricade, conspicuously post, and adequately control access to a high radiation area (HRA). Specifically, on September 17, 2009, a crane flagman on a Radiation Work Permit (RWP) that did not allow access to a HRA, inadvertently entered an unposted but guarded transient HRA and recorded an electronic dosimeter (ED) dose rate alarm at 128 mrem/hr. The worker was unable to hear the alarm due to wearing a headset and not wearing an auxiliary alarm device as specified in station procedures for HRA entries. The worker had been briefed to not enter the area when an irradiated instrument was on the floor and that the guard would prevent his entering the area. The guard was not positioned to prevent entry into the area and did not detect the flagman entering the area until he had already passed the source. The licensee entered this issue into their corrective action program as PIP M-10-05506.

The finding is greater than minor because it is associated with the cornerstone attribute of exposure control and affected the Occupational Radiation Safety Cornerstone objective of ensuring the adequate protection of the worker health and safety from exposure to radiation from radioactive material during routine civilian nuclear reactor operation because it resulted in unplanned or unintended radiation dose. The finding was determined to be of very low safety significance (Green) because it was not an ALARA finding or overexposure, did not have a substantial potential for overexposure, and did not compromise the ability to assess dose. The cause of the finding was directly related to the cross-cutting aspect of radiological safety in the work control component of the Human Performance area because the licensee did not adequately control the areas as a HRA. [H.3(a)] (Section 2RS1).

Inspection Report# : [2010003](#) (*pdf*)

Public Radiation Safety

Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Significance: N/A Oct 22, 2010

Identified By: NRC

Item Type: FIN Finding

2010 McGuire PI&R

The inspectors concluded that, in general, problems were properly identified, evaluated, prioritized, and corrected. The licensee was effective at identifying problems and entering them into the corrective action program (CAP) for resolution, as evidenced by the relatively few deficiencies identified by external organizations (including the NRC) that had not been previously identified by the licensee, during the review period. The licensee effectively used risk in prioritizing the extent to which individual problems would be evaluated and in establishing schedules for implementing corrective actions. Generally, prioritization and evaluation of issues were adequate, formal root cause evaluations for significant problems were adequate, and corrective actions specified for problems were acceptable. However, the inspectors identified several examples where issues were not prioritized in accordance with site CAP guidance and two examples of evaluations which lacked appropriate rigor. Overall, corrective actions developed and implemented for issues were generally effective and implemented in a timely manner.

The inspectors determined that overall, audits and self-assessments were adequate in identifying deficiencies and areas for improvement in the CAP, and appropriate corrective actions were developed to address the issues identified. Operating experience usage was found to be generally acceptable and integrated into the licensee's processes for performing and managing work, and plant operations.

Based on discussions and interviews conducted with plant employees from various departments, the inspectors determined that personnel at the site felt free to raise safety concerns to management and use the CAP to resolve those concerns.

Inspection Report# : [2010006](#) (*pdf*)

Last modified : March 03, 2011