

Vogtle 1

2Q/2010 Plant Inspection Findings

Initiating Events

Mitigating Systems

Significance:  Jun 30, 2010

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to verify purchased equipment conformed to design specifications

Green: A self revealing, non-cited violation of 10 CFR Part 50, Appendix B, Criterion VII, Control of Purchased Material, Equipment, and Services, was identified for failure to establish measures to assure that purchased material, equipment, and services conform to the procurement documents. More specifically that safety-related EMAX breaker closing coils were capable of performing their safety related function. All affected EMAX breaker closing coils were replaced with a qualified 90V closing coil capable of continuous duty cycle.

This finding is more than minor because if left uncorrected, the failure to establish measures to assure that purchased material, equipment, and services conform to procurement documents could become a more significant safety concern. Additionally, it impacted the Reactor Safety Cornerstones of Mitigating Systems and Barrier Integrity in that, the failure to establish measures to assure that purchased material, equipment, and services conform to procurement documents to ensure that safety-related breakers are assembled and functionally tested correctly, impacted the design control and equipment performance (availability and reliability) attributes. This finding was determined to be of very low safety significance (Green) because it did not result in a loss of operability or functionality. This finding was determined to not have a cross-cutting aspect associated with it due to the timeframe of the event and that the cause of the event is not indicative of current plant performance. (Section 1R18)

Inspection Report# : [2010003](#) (*pdf*)

Significance:  Dec 31, 2009

Identified By: NRC

Item Type: NCV NonCited Violation

Operation of NSCW system with tower return valves in open bypass

•Green. A NRC-identified NCV for failure to enter TS LCO 3.7.8 Condition A as required was identified.

Specifically, the licensee's failure to follow the requirements of TS LCO 3.0.2 and enter TS LCO 3.7.8 Condition A when the NSCW tower return valves are placed in a position other than that required by TS SR 3.7.8.1. The licensee has entered the issue into their corrective action program and began procedure revisions to ensure operation of the NSCW system in accordance with Technical Specifications and the UFSAR at all times.

This issue is more than minor because it is associated with a cornerstone attribute and adversely affected the objective of the Mitigating Systems cornerstone. Specifically, the performance deficiency is a configuration control error which affected the operability of an entire train of emergency core cooling system equipment, and thus impacts the equipment's automatic function to respond to a loss of coolant accident. The finding was determined to be of very low safety significance (Green) because the event did not represent an actual loss of safety function of a single train for greater than its Technical Specification allowed outage time. The inspectors determined that this issue does not have a cross-cutting aspect. The issue centers on differing interpretations of the Technical Specifications and the UFSAR, and does not align itself with any cross-cutting aspect. (Section 4OA5.1)

Inspection Report# : [2009005](#) (*pdf*)

Significance:  Dec 31, 2009

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate surveillance procedures for TS SR 3.7.8.2

•Green. A NRC-identified NCV for inadequate surveillance procedures was identified. Specifically, TS SR 3.7.8.2 requires the licensee to periodically verify that each NSCW system automatic valve in the system flow path actuates to the correct position on an actuation signal. The current procedures used to meet the requirements of TS SR 3.7.8.2 do not verify that the tower return header valves actuate to the correct position when demanded during an automatic actuation signal. As a result, the NSCW systems do not currently meet the requirements of TS SR 3.7.8.2. The licensee has entered the issue into their corrective action program and began procedure revisions necessary to support operation of the NSCW system in accordance with Technical Specifications and the UFSAR at all times.

This issue is more than minor because it is associated with a cornerstone attribute and adversely affected the objective of the Mitigating Systems cornerstone. Specifically, the performance deficiency is an equipment performance error which affected the reliability of the NSCW systems. The finding was determined to be of very low safety significance (Green) because the finding did not represent the actual loss of safety function of a single train for greater than its technical specification allowed outage time. The inspectors determined that this issue does not have a cross-cutting aspect. The issue centers on differing interpretations of the Technical Specifications and the UFSAR, and does not align itself with any cross-cutting aspect. (Section 4OA5.2)

Inspection Report# : [2009005](#) (*pdf*)

Significance:  Dec 31, 2009

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to properly maintain the approved fire protection program with regard to the relocation of the plant the fire alarm annunciation signal outside of the MCRs.

•Green. NRC identified a NCV of Vogtle Nuclear Plant Units 1 & 2 Operating License Condition 2.G, “Fire Protection,” for failure to properly maintain the NRC-approved fire protection program with regard to the location of the fire alarm computer audible and visual annunciation notification signal. Specifically, the licensee had implemented a plant change, in December 2006, for the fire alarm computer which relocated the fire alarm computer annunciation signal outside the continuously manned main control room to a clearance and tagging office which was not continuously manned. The plant change could have resulted in a delay of up to 2 minutes before the alarm would have been relayed to the main control room for actions to dispatch the fire brigade and initiate safe shutdown actions.

The finding is more than minor because it is associated with the reactor safety, mitigating systems, cornerstone attribute of protection against external factors, i.e. fire, and it affected the objective of ensuring reliability and capability of systems (i.e., fire detection) that respond to initiating events. The finding was determined to be of very low safety significance (Green) in a Significance Determination Process Phase 1 analysis because the two minute delay had only minimum impact on the feasibility or reliability of the time critical operator actions and fire brigade performance in response to a fire. This violation was entered into the licensee’s corrective action program as Condition Report 2007110797. No cross cutting issue was identified, because the finding is not indicative of current plant performance. (Section 4OA5.3).

Inspection Report# : [2009005](#) (*pdf*)

Barrier Integrity

Significance:  Sep 30, 2009

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

MOV program procedures were inadequate with regard to periodicity of preventive maintenance activities for stem lubrication

A self-revealing NCV of 10 CFR 50, Appendix B, Criterion V,

“Instructions, Procedures, and Drawings,” was identified. Specifically, Vogtle Electric Generating Plant’s (VEGP) MOV preventative maintenance (PM) procedures lacked specific instructions that provided an adequate frequency for performing valve stem lubrication, which resulted in test failures of safety-related MOVs and affected the reliability of the MOVs’ safety functions. The licensee removed the hardened grease, re-lubricated and successfully tested the MOVs. They have entered the issue into their corrective action program and are in the process of revising existing maintenance procedures to change the PM frequency from 54 months to 36 months for long stem, safety-related MOV stem lubrication.

The finding was more than minor because if left uncorrected other safety related MOVs could be affected by the inadequate stem lubrication PM frequencies. The finding is associated with the configuration control attribute of the Barrier Integrity (BI) Cornerstone and affected the cornerstone objective of providing reasonable assurance that physical design barriers (e.g., containment) protect the public from radionuclide releases caused by accidents or events. Specifically, Containment Spray (CS) pump sump suction isolation MOVs experienced test failures and were declared inoperable, which required operability evaluations, thereby challenging their reliability and capability to perform their safety function. Using the Phase 1 worksheet in Attachment 4 of Manual Chapter 0609, “Significance Determination Process,” the finding affected the BI cornerstone and was of very low safety significance (Green) because it did not represent an actual open pathway in the physical integrity of reactor containment. Although the CS sump suction MOV’s condition affected the mitigating system cornerstone, the finding analysis was assigned to the BI cornerstone because it best reflected the dominant risk of the finding. This finding has a cross-cutting aspect in the area of PI&R, Corrective Action Program, because VEGP did not thoroughly evaluate problems such that the resolutions addressed the causes and extent of condition [P.1(c)]. Specifically, VEGP failed to thoroughly evaluate previous conditions of degraded and hardened grease on safety-related valves, such that the extent of the condition was considered and the cause was resolved.

Inspection Report# : [2009004](#) (*pdf*)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Last modified : September 02, 2010